

ST Reactor – new considerations

“Re-thinking of the Fusion Power Plant concept”

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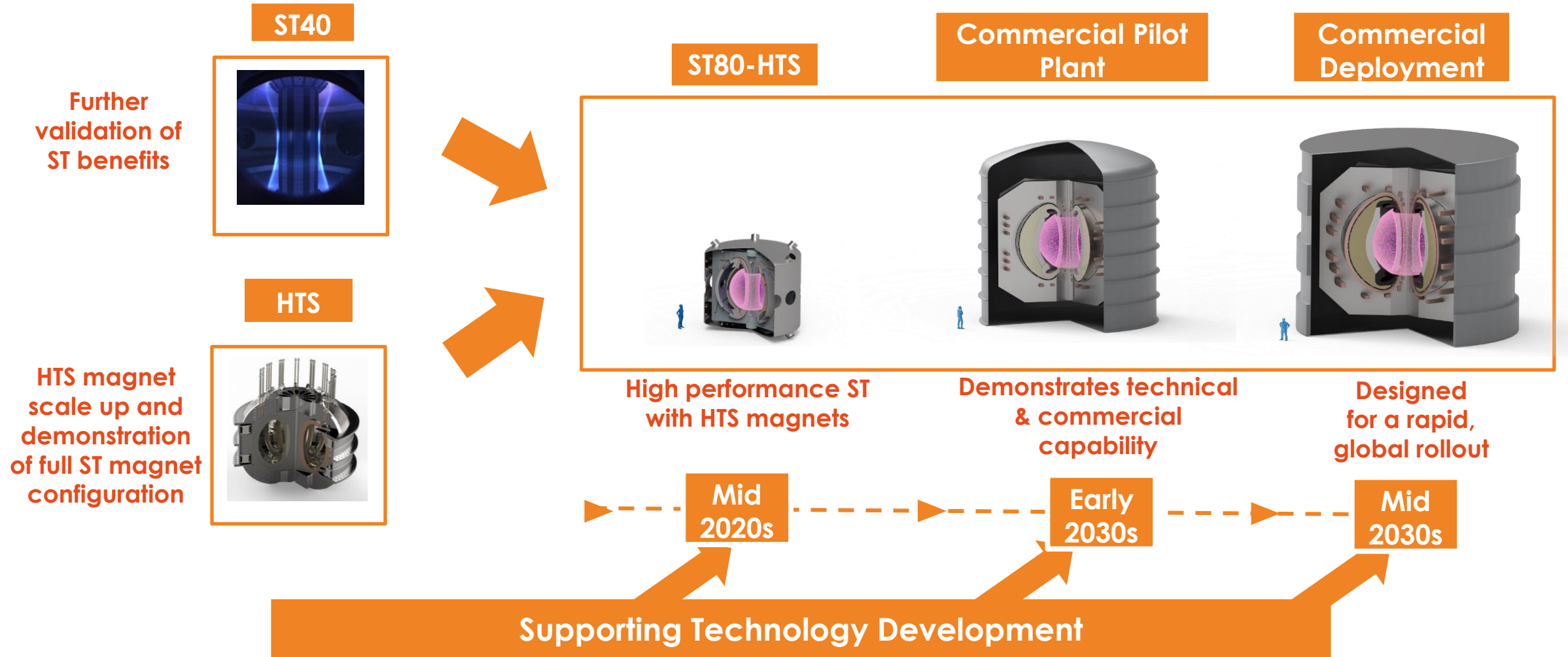
Remote Meeting

Innovations in Physics and Technologies are the basis for a **new approach** to the design of a **Fusion Reactor**

- What is the fastest path to a **commercial** Fusion Reactor?
- Up to now, designs of Fusion Reactors were based on Physics and Technologies of the last century. **New ideas** are emerging.
- Our approach has many **common ground with** mainstream Tokamak Fusion (e.g. **ITER**, DEMO, STEP). We rely on the **same physics** behind the magnetic fusion concept ... but we have a **faster way** to get to a commercially viable device.



The realisation of commercial fusion



ST80-HTS Mission

- Demonstrate the successful integration of many of the critical engineering technology building blocks.
- Demonstrate long pulse operation with high duty cycle, at high plasma performance



Four Main Innovations:

1. Spherical Tokamaks

Squashed shape, compact

Highly efficient, *high β* :

*from 12% in DIII-D to 40%
in START/NSTX*

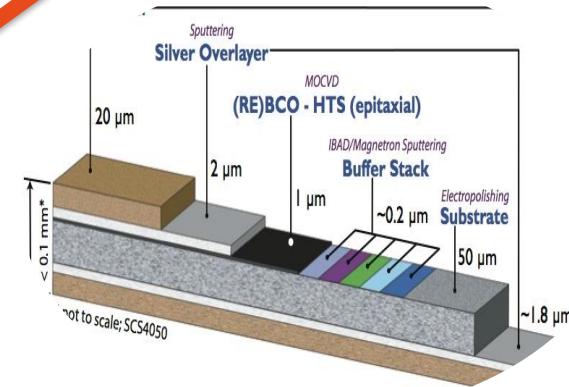


2. High Temperature Superconductors

High field

Quench protection simplified

Lower cryogenic cooling requirements



smaller, cheaper, faster

3. Li technologies

As a path to low recycling regime
and sustainable divertor solution

4. Innovative Physics:

new advanced confinement regimes



Some other ideas to boost tokamak performance

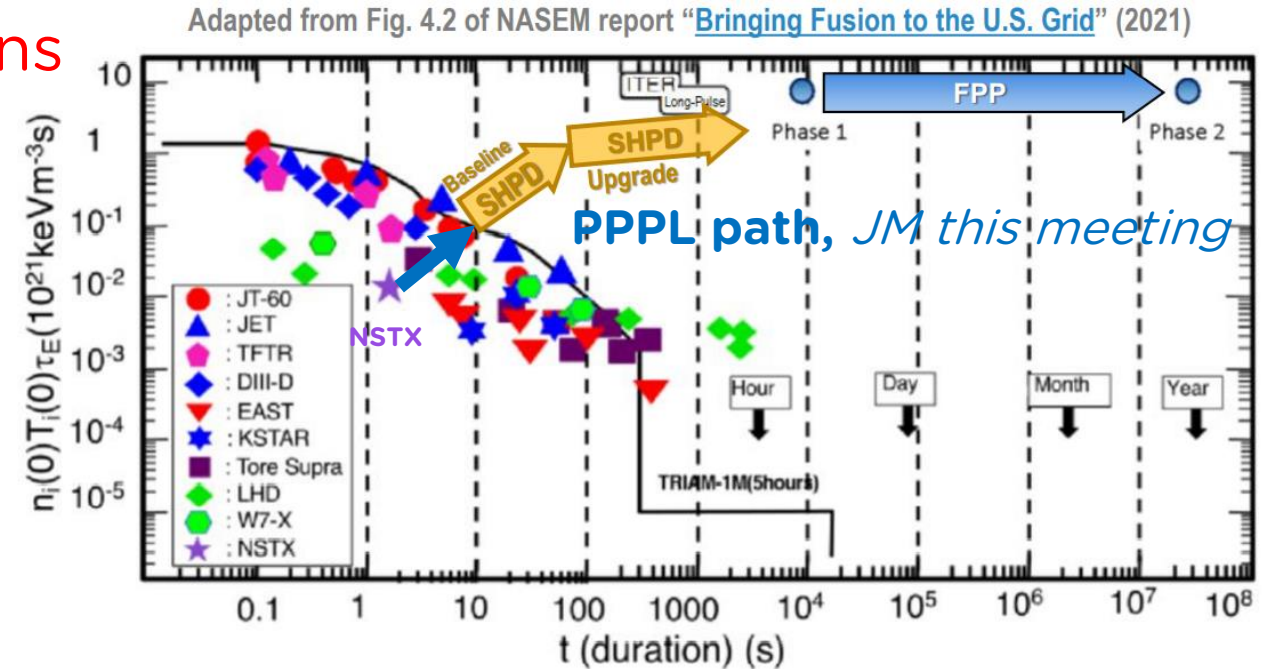
- To-day's Fusion reactor concept is based on the thermal DT Fusion and 20th century physics (H-mode). Are there other ways? **Results of recent experiments** are promising:
 - **Low recycling regimes and Hot Ion mode** are now considered as one of promising operation scenarios to get burning conditions in a fusion reactor (FIRE mode, KSTAR, *Nature* 2022, *SOFT* and *APS* 2022).
 - Hot Ion mode has been also demonstrated in STs: ST40 (Asunta, Romanelli, S.M. Kaye, this meeting), Globus-M2 (Zhiltsov, this meeting) and suggested on other STs (EHL-2, Huasheng Xie, this meeting)
 - Recent JET experiments also show big advantages of the **low recycling hot ion mode** regime (J.Garcia et al, Ph Pl 2021)
- **Beam-plasma** and beam-beam reactions are known to contribute to Fusion power
 - In big and medium size tokamaks this contribution is known and well determined (~50% on JET, more on MAST/NSTX and ST40)
 - Can we boost Q_{fus} optimising **beam-plasma**? And use fast particles for stability improvement (FIRE-type stabilisation, sawteeth stabilisation)



Reality. Are long pulses possible? Not up to now.

Challenges with steady-state operations

- there **are no steady-state tokamaks or stellarators** with reactor relevant plasma parameters
- There are **no proper experimental facilities** since devices for such studies are **very expensive and risky**.



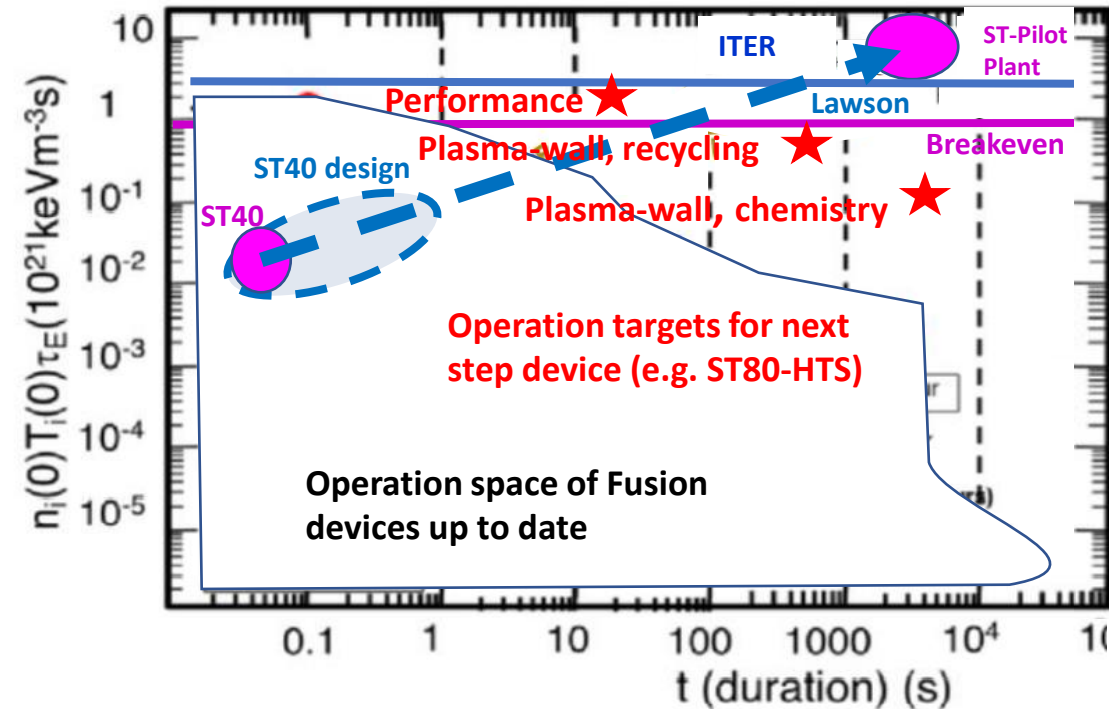
- Although technology issues associated with steady-state magnets, heating and CD auxiliary systems, and requirements for their maintenance could be resolved, there is growing evidence that the **duration of the burning pulse may be limited by other factors**.

- Even in relatively short pulses unexplained fast disruptions have been observed which can be attributed to **dust** generation, **flaking** of depositions on the **vessel wall** and **central post**, formation of **hot spots** and other plasma-wall interaction phenomena.



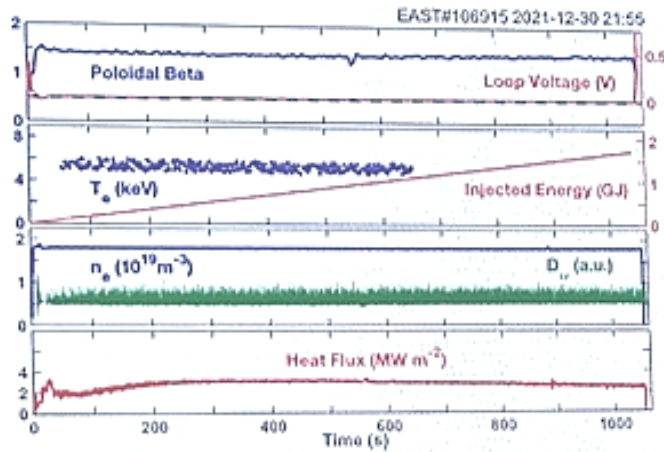
Main timescales for performance limitations

- Several timescales can be considered for possible **pulse duration limitation**:
 - **Fast** due to dust, flaking, hot spots, VDE, MHD
 - Recycling is much slower, from tens of seconds to - **5/15min** w/o / with Li conditioning (*HIDRA, APS 2022*)
 - Accumulation of **chemistry** on and in the wall (**30 - 40 min**)
- These timescales determine necessary pulse durations in next step experimental devices and test beds to find recipes for mitigation and achievement of long discharges in future devices*



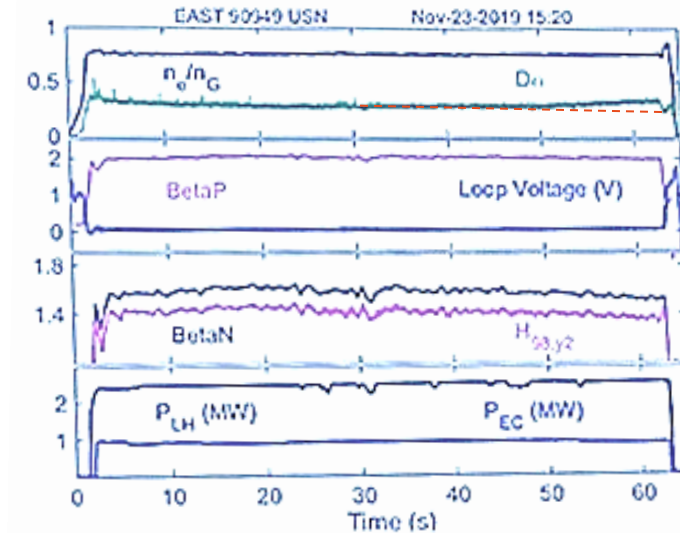
- Engineering and **thermal cycle** (blanket, divertor etc) **timescales** should also be taken into account

Are **longer** pulses possible? Yes (*SOFT, APS 2022*).



$T_e(0) \sim 6.5 keV$, $T_i(0) \sim 1.0 keV$, $n_i(0) \sim 2.2 \times 10^{19} m^{-3}$, $\tau_E \sim 60 ms$

- Fully non-inductive plasma with dominant e-heating
 - Zero injected torque
 - $P_{HJ} \sim 1.65 MW$ ($P_{UH} \sim 1.1 MW$, $P_{ECH} \sim 0.55 MW$) with $E_{HJ} \sim 1.73 GJ$
 - $H_{92} \sim 1.3$
- Stable control of plasma equilibrium configuration
- Good control of particle recycling and heat load



EAST 1056 sec discharge **w/o** expected recycling or chemistry **limitation**

- The secret is “Control of recycling”, i.e. Li conditioning

But **EAST H-mode** shot was limited to 60 sec.

- Note “hardly visible” increase in D_{10} around 50 sec
- **KSTAR** performance degradation was mitigated with reducing wall recycling due to injection of Boron Nitride, hot vessel, wall conditioning.
- Boronisation resulted in significant increase in pulse duration on **W7X**

- But we need to demonstrate long pulses at **10 – 100 times higher performance ($nT\tau$)**



Could pulsed pilot plants overcome long pulse challenges?

- Recent studies (*Segal et al, NF 2021*) have shown potential advantages of a **pulsed** fusion reactor over a conventional **steady-state** reactor:
 - **reduced** need of very expensive non-inductive **current drive**;
 - lower risk, as the steady state operations may have limitations;
 - broader range of possible plasma parameters (not limited by the CD requirements or H-mode, *Frank et al, NF 2022*);
- **Pulsed ST reactors** have even more advantages due to:
 - possibility of **high bootstrap current**,
 - stronger increase in the **confinement** with the toroidal field,
 - good stability,
 - lower requirements on the volt-second capability of the central solenoid for the plasma current sustainment.



How long should be burning phase in a Reactor?

- Reactor efficiency criterion *E Rebhan, van Oost, Fusion Science and Technology 2017* :

$$\frac{\tau_b}{\tau_E^*} = \frac{(1 + \tau_h/\tau_{E,h}^*)}{5\eta_a\eta_{\text{eff}}(1 - \eta/\eta_{\text{eff}})},$$

τ_b is burn time, η is efficiency of power station, $\eta_{\text{eff}} \sim \eta_{\text{th}} \sim 1/3$, $\eta_a\eta_{\text{eff}} \sim 1/20 \Rightarrow \tau_{\text{burn}} \sim \mathbf{160\tau_E}$

- **Burn time of 2h** would allow 100 000 cycles for 25 years of reactor life time for typical conventional reactor with $\tau_E \sim \mathbf{3.5 \text{ sec}}$

- Another estimate *Segal et al, NF 2021*:

- 500 cycles per month, assuming 15000 maximum N_{cycle} before SC replacement, or

$$\tau_P = 720 \frac{N_{\text{ON}}\tau_{\text{OFF}}}{N_{\text{CYC}}} = 1.44 \sim 1.5 \text{ hours per pulse}$$

- 1 – 2 h pulses are assumed for most of today's DEMO proposals
- ST reactor** may be smaller and confinement time $\sim 1 \text{ sec}$, so needed pulse (burn) duration may be just 30 – 60 min

Could pulsed pilot plants overcome long pulse challenges?

- For these pulse durations, **central solenoid** can be the most efficient tool to sustain plasma current, replacing expensive RF CD
 - Steady state reactor needs about 50% higher fusion gain than a pulsed reactor due to its low RF conversion efficiency (*Segal et al, NF 2021*)
 - However, **non-inductive** methods should be used for plasma **current ramp-up** (H Idei, N Tsujii, Y Ono, M Ono, K Kuroda, M Inomoto, E. du Toit, H. Tanaka, K. Iwasaki, N. Tsujii, S.J. Diem, Tan Yi, etc this meeting) **to save solenoid flux for the flat-top**
- Pulsed operations may allow solutions for chemistry treatment between shots (e.g. GDC, boronisation, Li conditioning and re-conditioning)
- Steady-state energy production can be achieved by **keeping blanket temperature high** enough between plasma pulses
 - Typical **blanket** full **cooling time** ~ 100 sec, assuming 20 – 30 sec needed to recharge solenoid, loss of CoE is not substantial. Only 3 MW needed for EU DEMO to keep blanket hot (SOFT 2022)



Advantages of an ST pulsed pilot plant

- The analysis described in (*Segal et al*) has been extended to STs. It can be concluded that pulsed reactors have some extra favourable properties, including :
 - Broader range of possible plasma parameters (not limited by the CD requirements);
 - Potentially simpler, faster, and cheaper development path as operations with the CS are well established;
- These advantages are even **more pronounced in STs** due to several specific features:
 - Possibility to operate at a higher beta;
 - Possibility to achieve a higher bootstrap current fraction;
 - Stronger increase in the confinement with the toroidal field;
 - Better stability, both for micro-pressure-driven micro instabilities and for the current-driven instabilities.
 - Lower requirements on the volt-second capability of the CS due to the ability to reach full plasma current without using the CS.



Further option – recharging of SC during steady-state

- Although the bootstrap current in an ST is expected to contribute up to high levels of ~90% of the total plasma current at the flat-top, the remainder (~10%) must be driven by other means and the **CS is an efficient** solution.
- Given that the CS is used only for the I_p sustainment and **not for the I_p ramp-up**, the **needed CS volt-seconds can be much smaller**.
 - When the limited CS volt-second capability is exhausted during the burning stage, it may be **recharged without plasma termination**.
 - Keeping plasma current constant during CS recharge significantly **reduces** engineering **challenges** from the **thermal cycling** of pulsed operation.
- RF and microwave auxiliary systems that were used during the ramp-up phase can be **re-applied during recharging**.
 - This will reduce the efficiency of the energy production because of the increased re-circulating power, but only for a considerably short period.



Recharging of SC during steady-state

- If needed, solenoid may be recharged during the pulse using combination of RF CD schemes that were used during ramp-up
 - ST has big advantages on this route - lower inductance, high bootstrap fraction
 - High neoclassical resistivity controversially helps

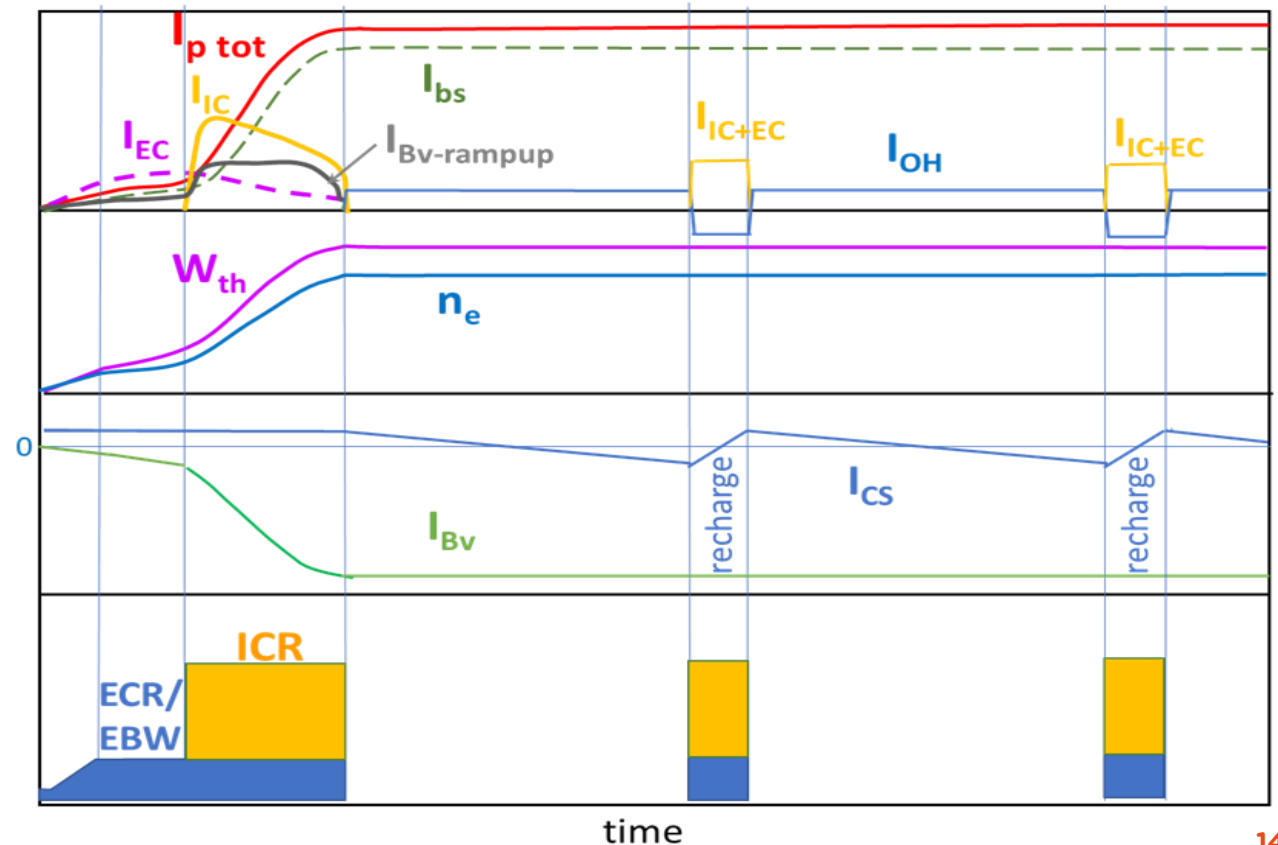
Proposed time sequence:

$I_{p \text{ tot}}$ - total plasma current and its components: I_{EC} , I_{IC} , I_{bs} , and $I_{Bv-rampup}$ (not to scale);

W_{th} - plasma thermal energy and n_e - electron density;

I_{CS} - CS current and I_{BV} - vertical field coil current;

and auxiliary CD and heating power application periods (schematic).



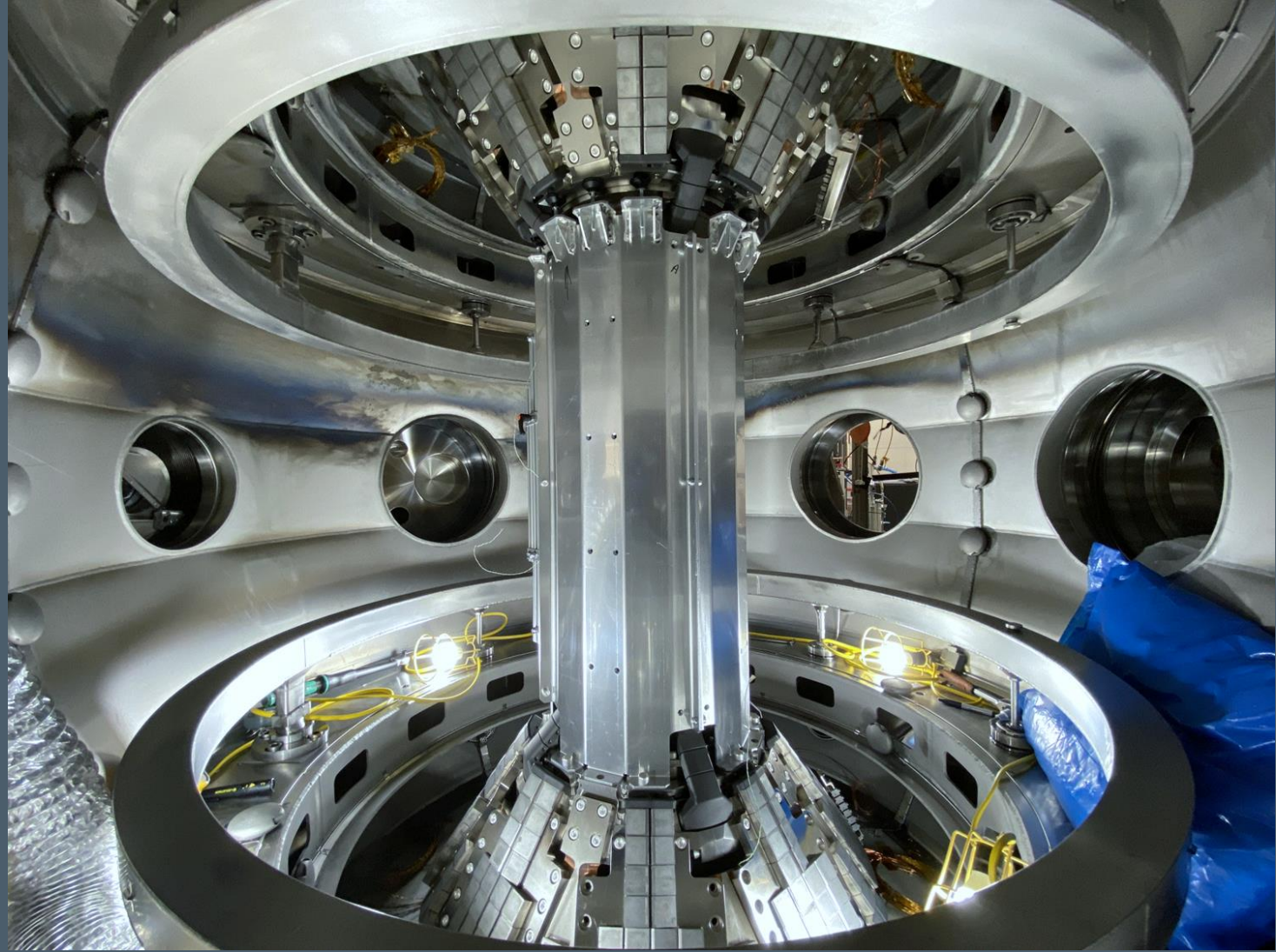
Conclusions

- Recent advances in Physics and technologies allow **re-thinking** of **Fusion Power Plant** concepts
- With the new approach, a pulsed device can be technically and economically feasible
- An ST **Fusion Pilot Plant** that has **competitive advantages for commercialization and cost reduction** and a strong economy of scale, leading to a **Power Plants** that are still small on an absolute scale



ST40: World Firsts

- Achieving a plasma temperature of 100 million degrees Celsius, the threshold
- This is by far the highest temperature ever achieved in a spherical tokamak and by any privately funded tokamak
- The triple product achieved is the highest by a private fusion energy company in a spherical tokamak with a plasma volume of less than one cubic meter
- This is 15 times less volume than any other tokamak which has achieved this temperature.



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